

# Fission Materials Overview

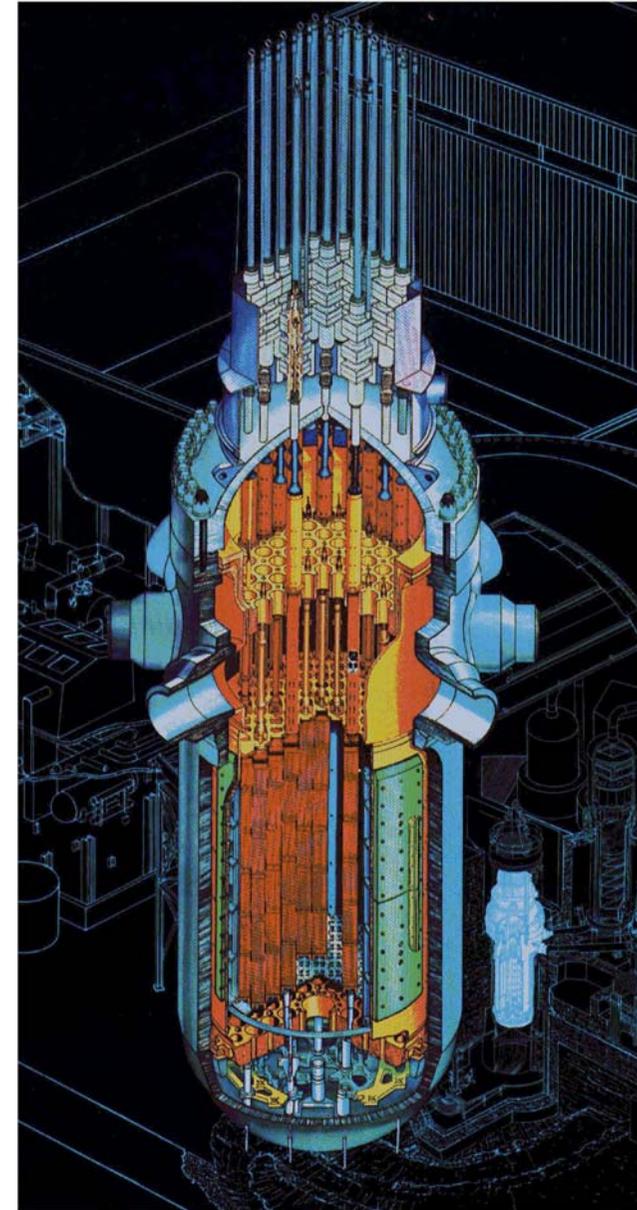
Andrew H. Sherry

“Energy Materials: Meeting the Challenge”  
Loughborough University, U.K.  
9 - 10th October 2008

# Introduction

## The challenge

- Energy policy has shifted significantly in recent years with new nuclear power emerging as a key component of the UK energy mix
  - *Security and diversity of supply*
  - *Low carbon*
- Government support for new nuclear build in 2008
  - “... the Government has today concluded that nuclear should have a role to play in the generation of electricity, alongside other low carbon technologies”
- This has been followed by the creation of the Office for Nuclear Development (OND)
  - To enable operators to build and operate new nuclear power stations in the UK from the earliest possible date*
- The National Skills Academy for Nuclear
  - To create, develop and promote world class skills and career pathways to support sustainable future for the UK nuclear industry*



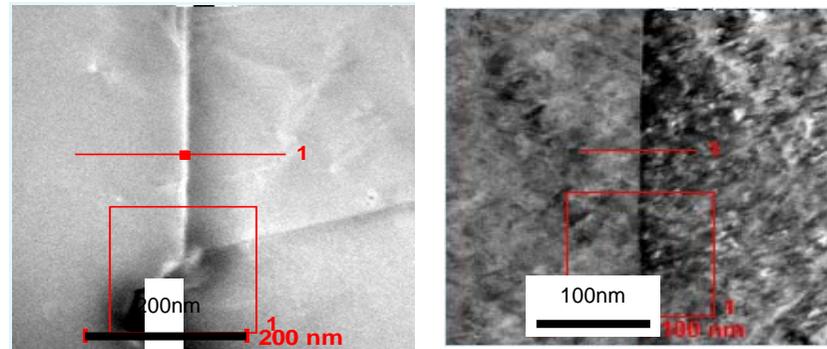
# Introduction

## The Grand Challenge

- Understanding and predicting materials performance in the extreme environments of nuclear applications is fundamental to support the nuclear renaissance
  - Fuels
  - Fuel cladding
  - Core materials
  - Pressure Vessels
  - Steam Generator
  - Pressuriser
  - Pumps
  - Pipes and welds
- High temperatures, irradiation fields and corrosive environments

Understand and predict changes in:

- microstructure )
- properties ) Safety and reliability
- loading )



20/25 stainless steel fuel cladding before and after proton irradiation

# AGR Plant Life Extension

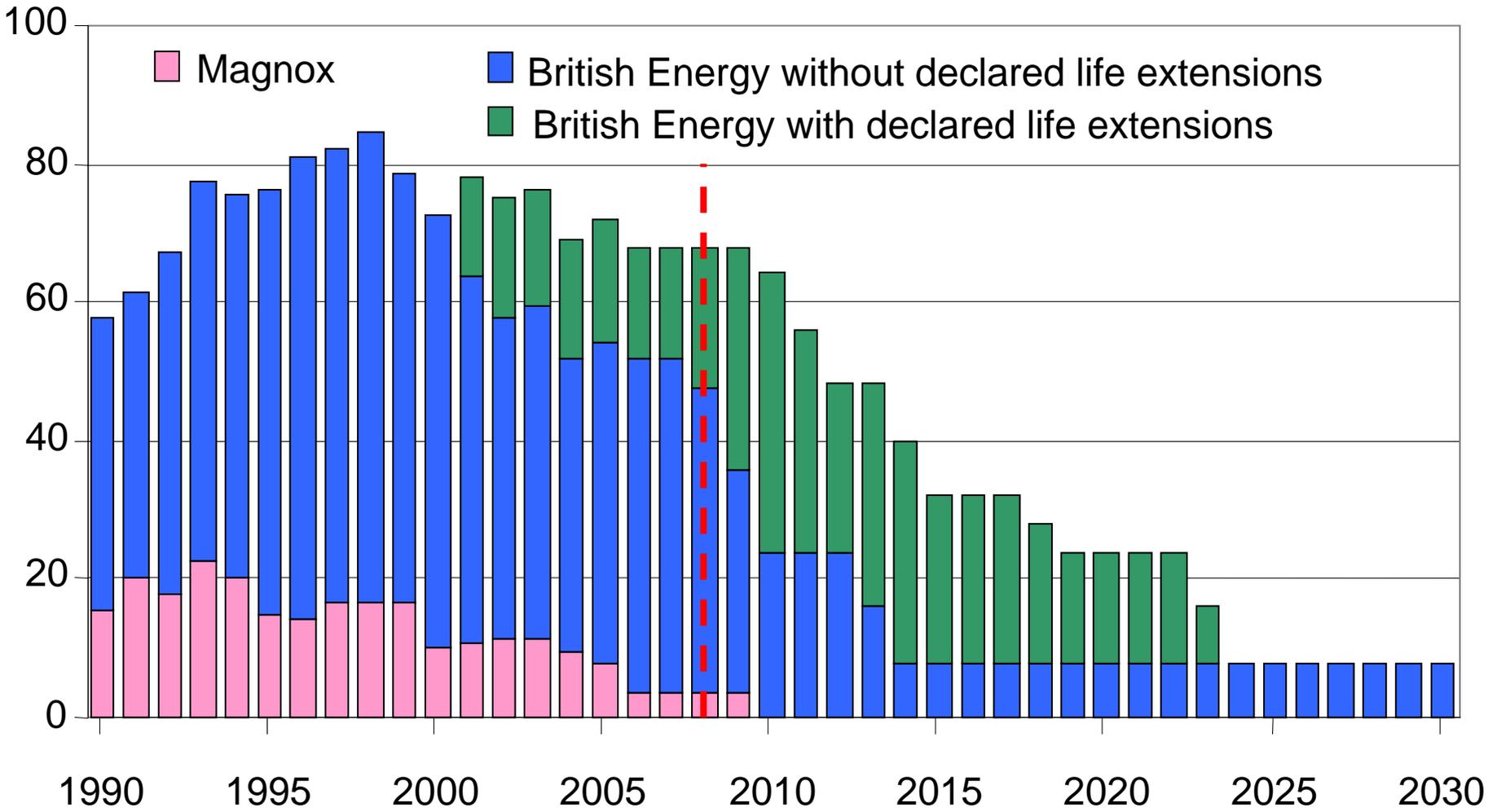
## British Energy's AGR Reactors

(Construction Dates)

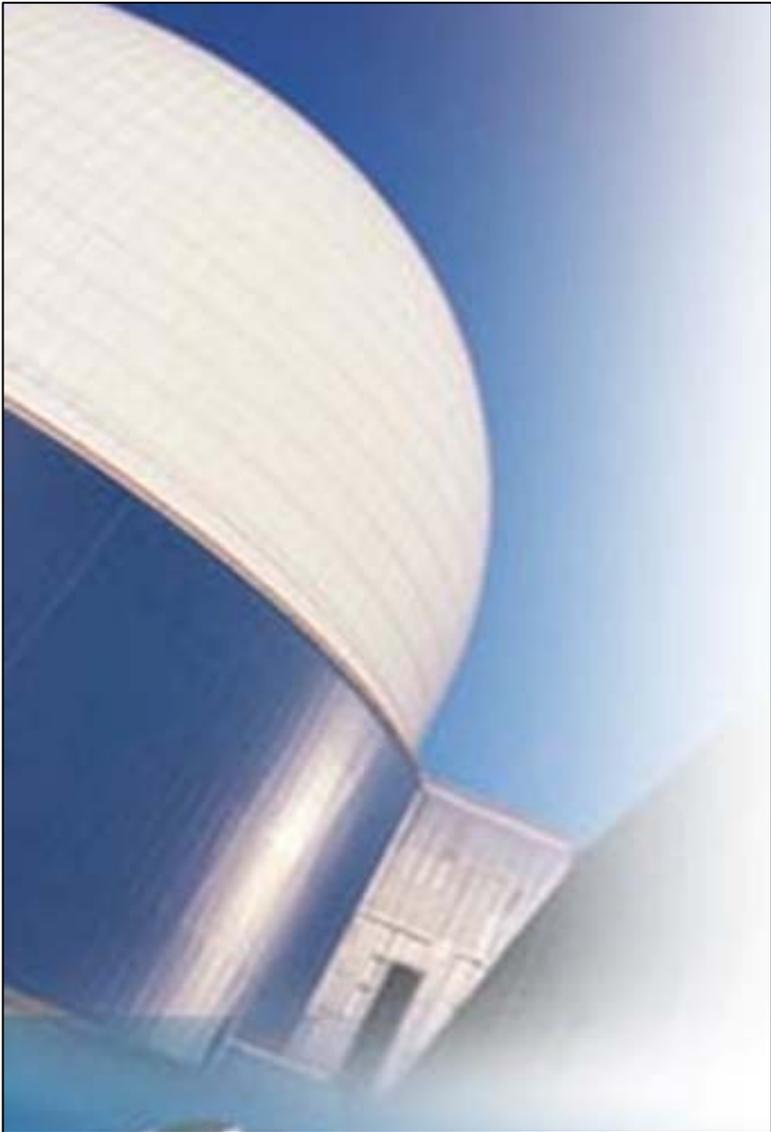


# AGR Plant Life Extension

Annual generation (TWh)



# PWR Operation, Life Extension and New Build



**HSE** Health and Safety Executive

**Public Report on the Generic Design Assessment of New Nuclear Reactor Designs**

AREVA NP SAS and Electricité de France SA UK EPR Nuclear Reactor

Conclusions of the Fundamental Safety Overview of the UK EPR Nuclear Reactor (Step 2 of the Generic Design Assessment Process)



**HSE** Health and Safety Executive

**Public Report on the Generic Design Assessment of New Nuclear Reactor Designs**

Westinghouse Electric Company LLC AP1000 Nuclear Reactor

Conclusions of the Fundamental Safety Overview of the AP1000 Nuclear Reactor (Step 2 of the Generic Design Assessment Process)

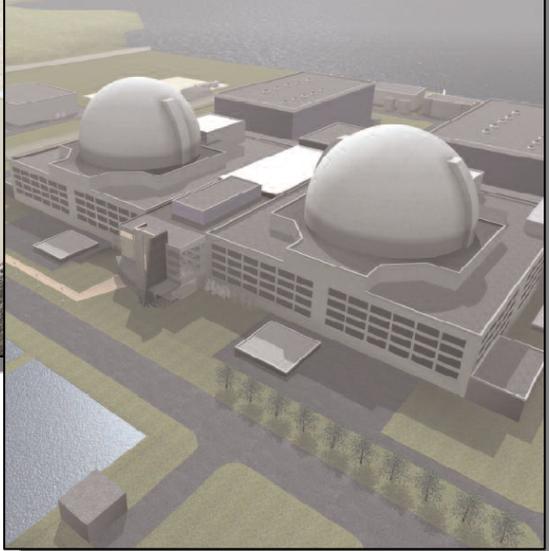


**HSE** Health and Safety Executive

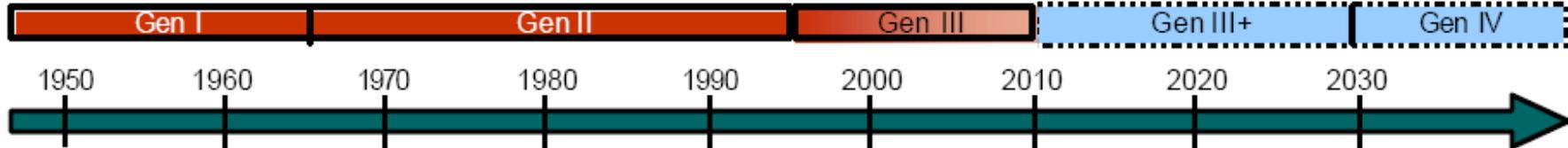
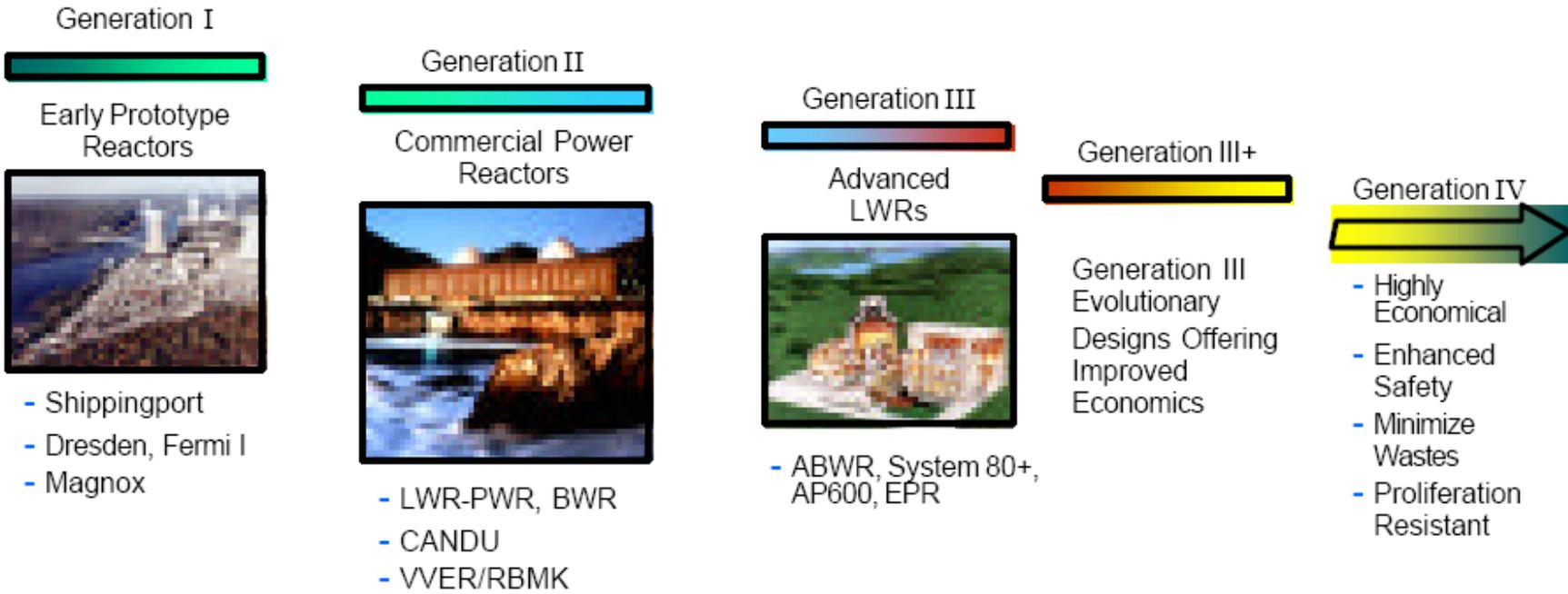
**Public Report on the Generic Design Assessment of New Nuclear Reactor Designs**

Atomic Energy of Canada Limited ACR-1000 Nuclear Reactor

Conclusions of the Fundamental Safety Overview of the ACR-1000 Nuclear Reactor (Step 2 of the Generic Design Assessment Process)



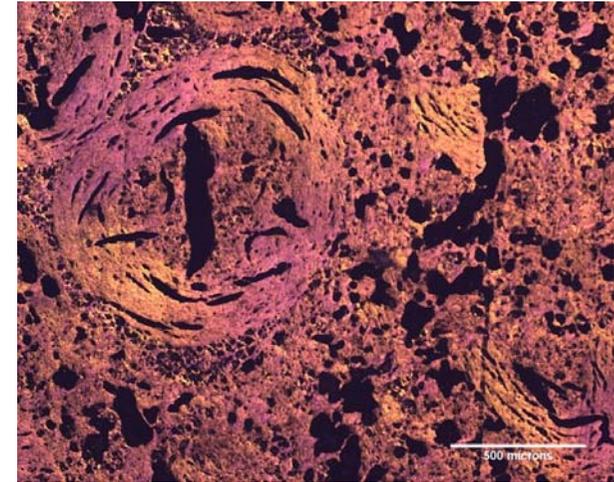
# Generation IV Reactor Designs



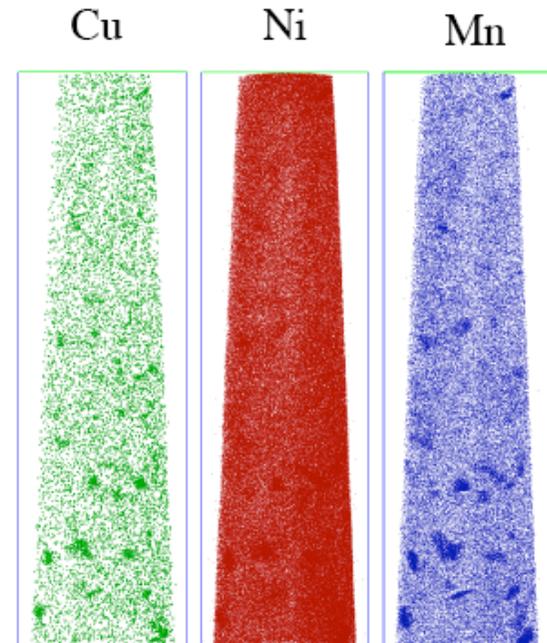
# Fission Materials

## The fission materials challenge

- Materials operate in extreme environments
  - High temperatures
  - Irradiation field
  - Corrosive environments
- Microstructural changes in service from the atomic scale upwards influence component performance
- The regulatory environment within the UK is non-prescriptive
  - requires a mechanistic approach to predict component

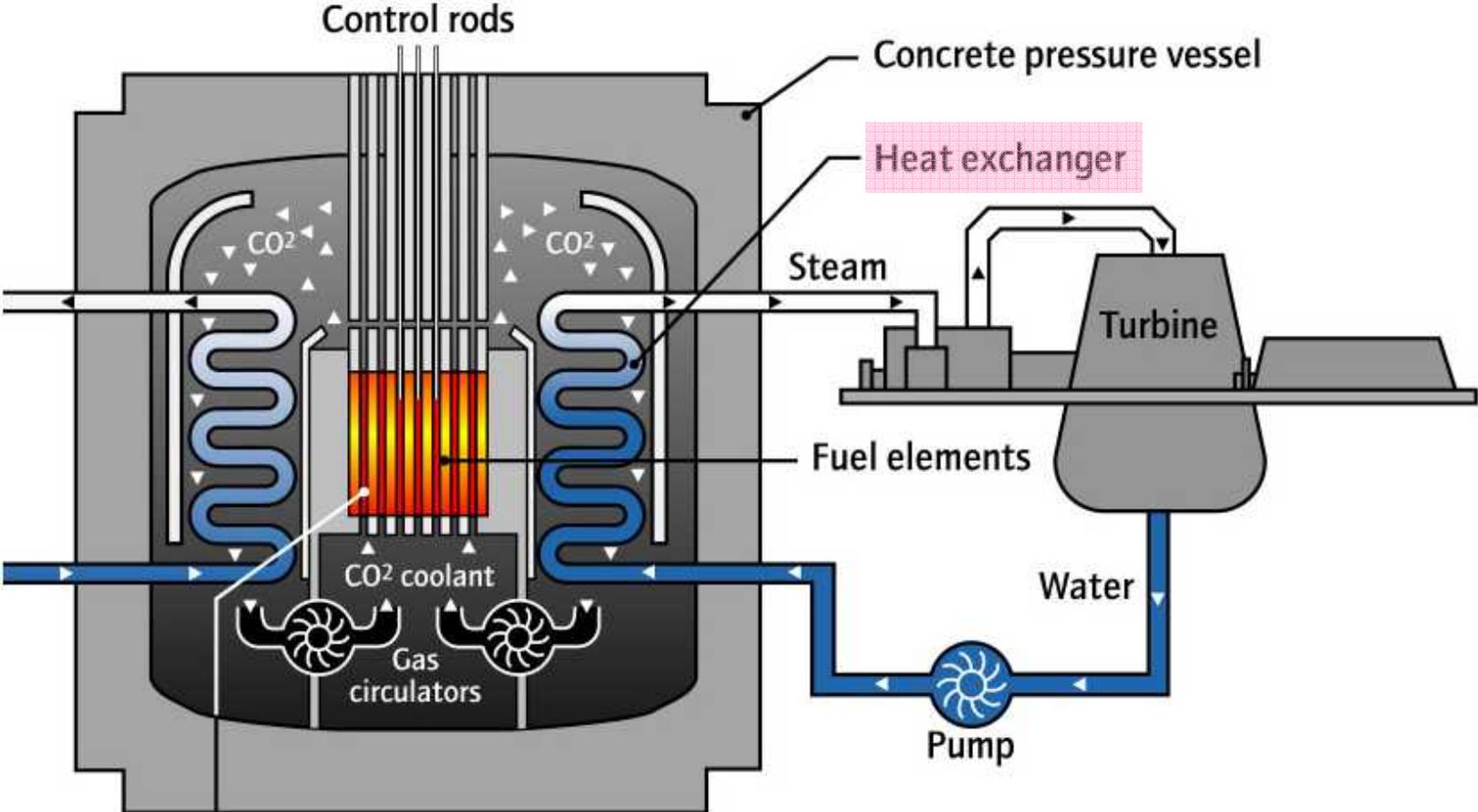


Microstructure of AGR graphite



Cu-rich clusters observed in 3D Atom Probe, Odette, 2004

# AGR Plant Life Extension

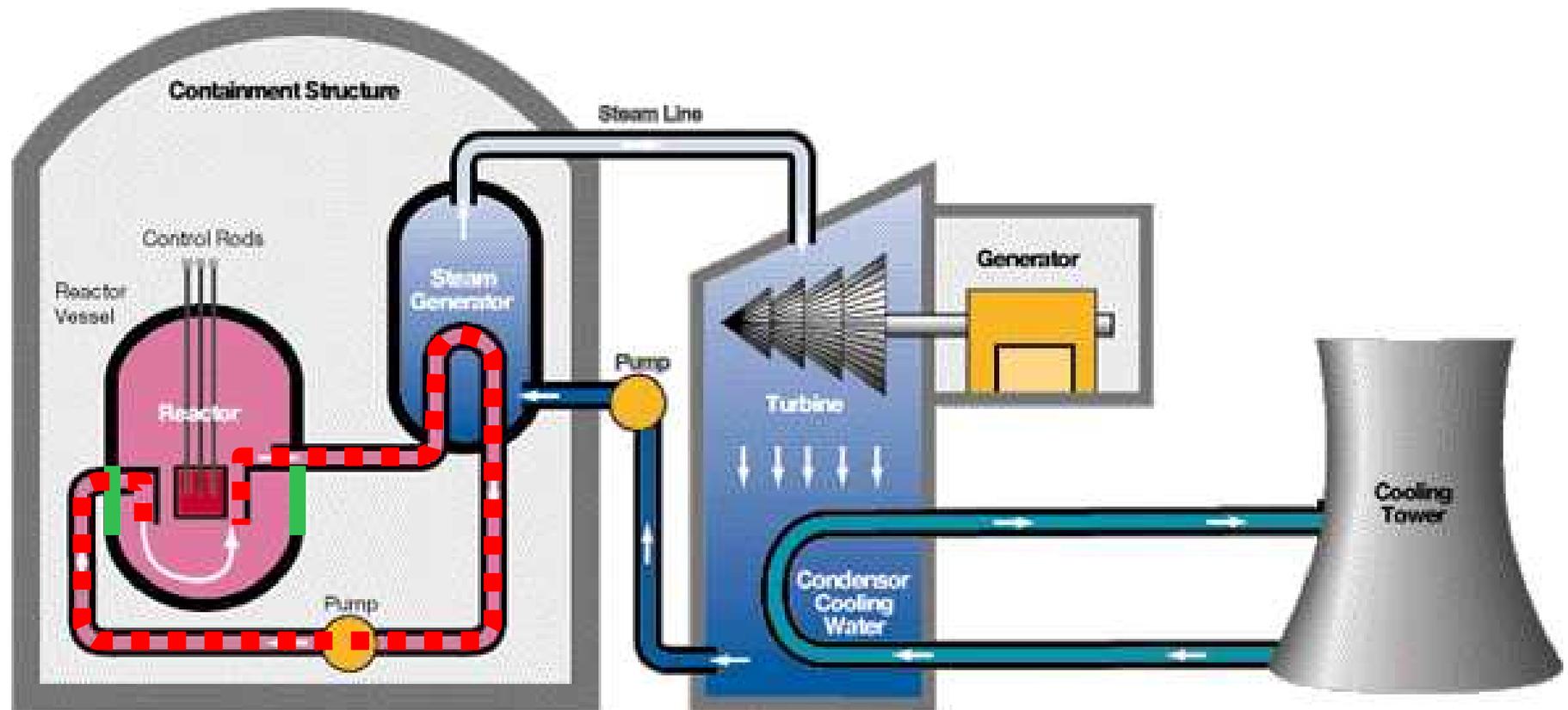


Graphite moderator

1. Degradation of graphite core

2. Degradation of high temp. welds

# PWR Plant Issues



3. Irradiation embrittlement of RPV

4. Environmentally assisted cracking

# Fission Materials

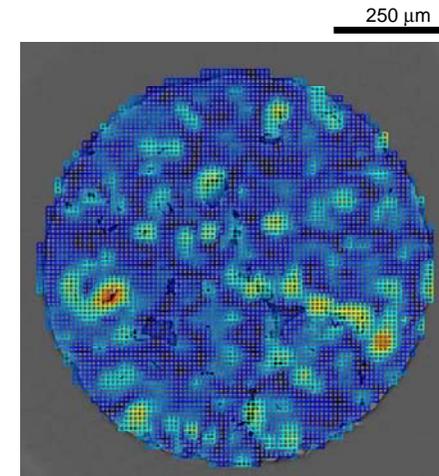
## The fission materials challenge

	High Temperature	Neutron Irradiation	Oxidation / Corrosion
AGR	✓	✓	✓
AGR	✓	✗	✓
PWR	✓	✓	✗
PWR	✓	(✓)	✓

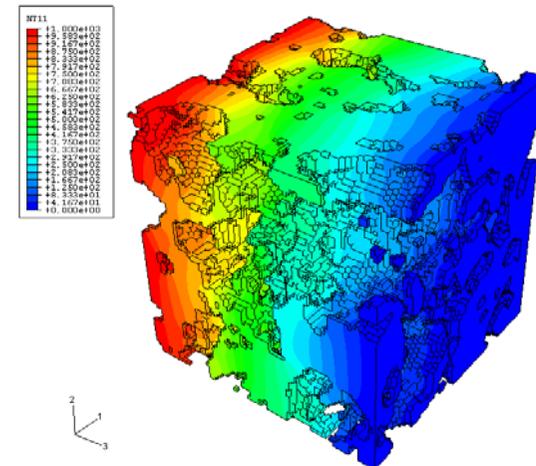
# AGR Plant Life Extension

## 1. Degradation of AGR graphite core

- During service the microstructure, properties and stress state of the graphite core change due to the combined effect of:
  - Fast Neutron irradiation  
Changes in dimension, physical and mechanical properties
  - Radiolytic oxidation  
Changes in density (weight loss), physical, elastic properties and strength
  - Irradiation creep reduces internal stresses generated by dimensional change



Distribution in thermal expansion within AGR graphite (image correlation + tomography)

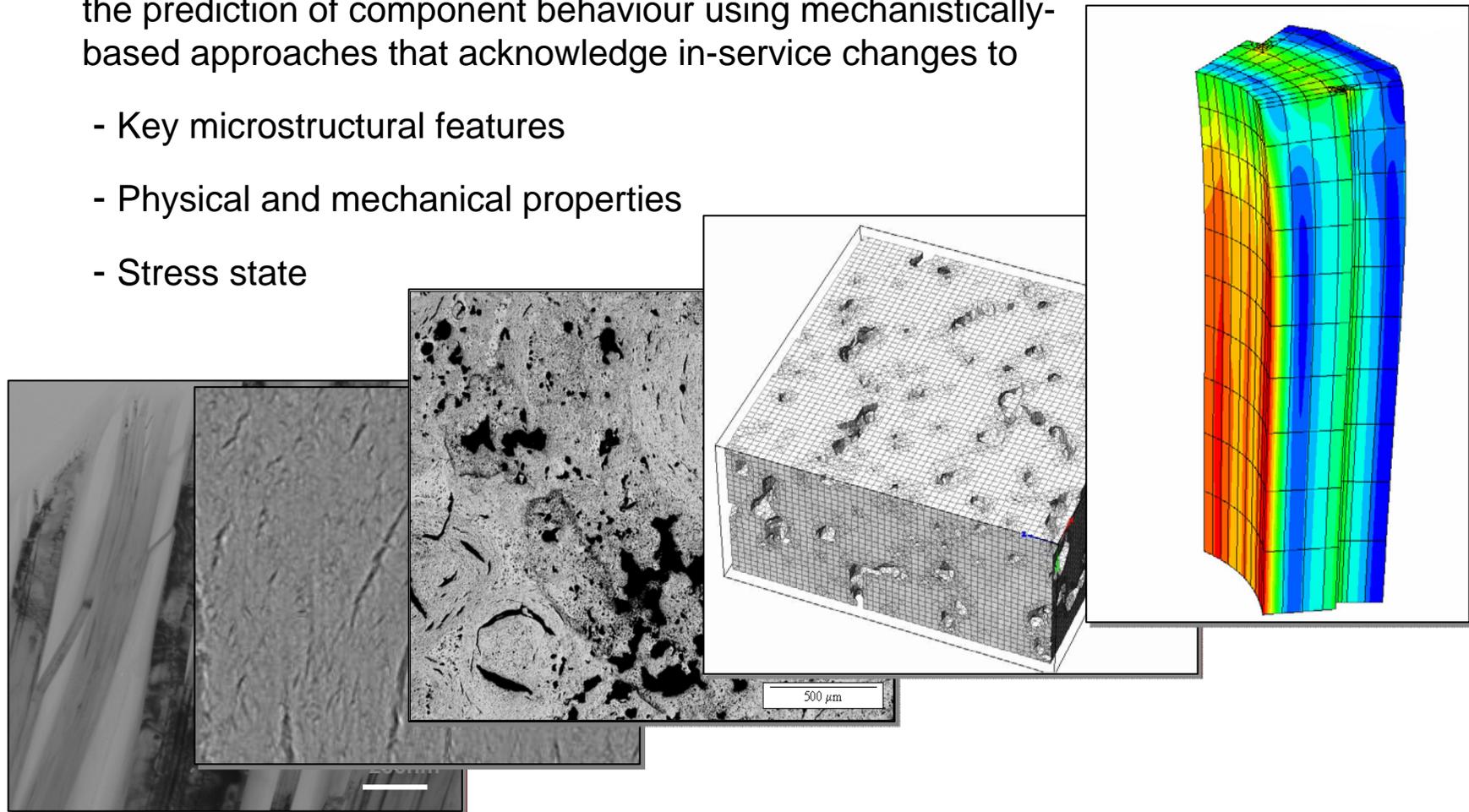


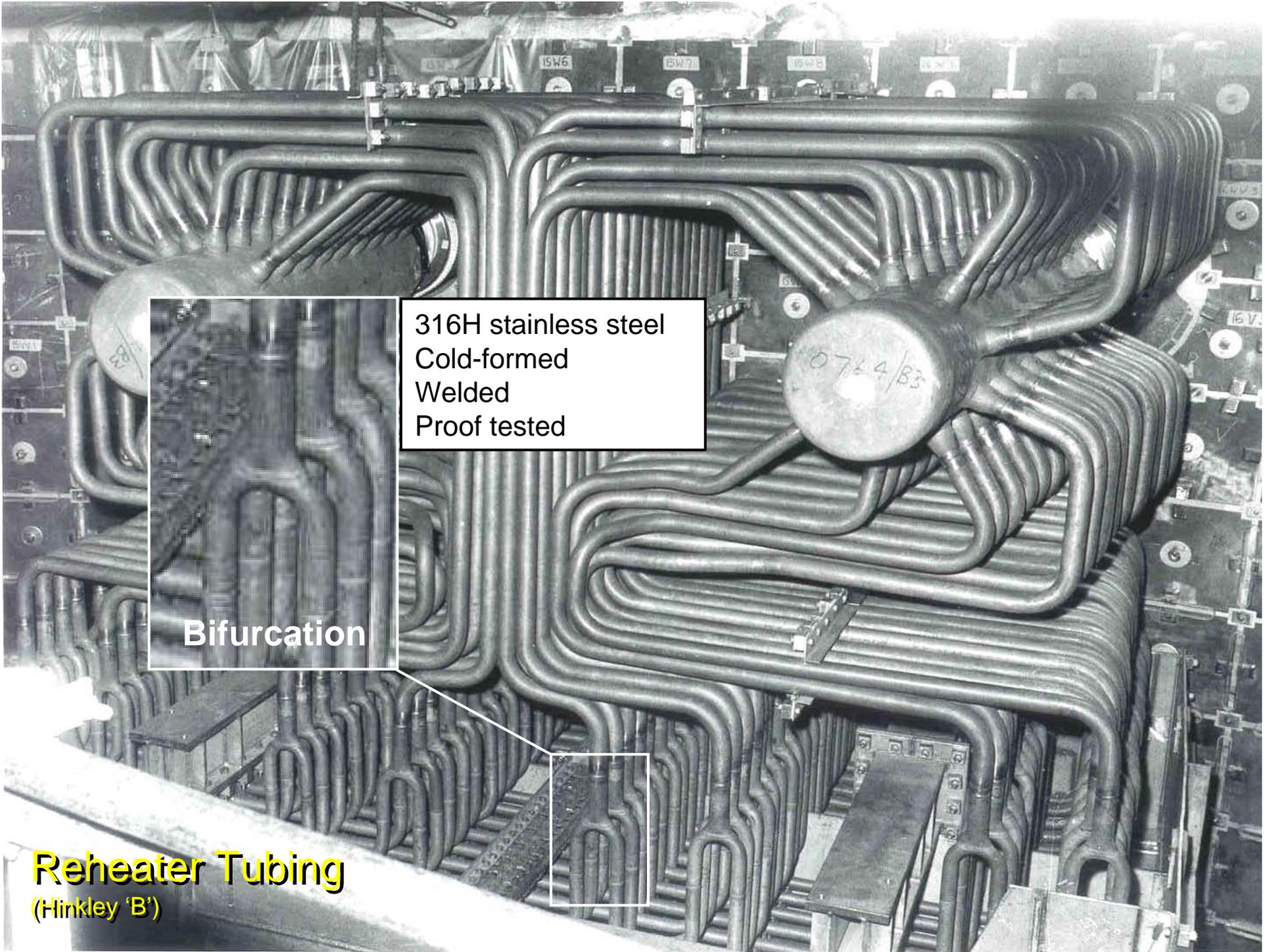
Predicted temperature distribution in AGR graphite sample (tomography + FEA)

# AGR Plant Life Extension

## 1. Degradation of AGR graphite core

- The materials challenge includes the prediction of component behaviour using mechanistically-based approaches that acknowledge in-service changes to
  - Key microstructural features
  - Physical and mechanical properties
  - Stress state





316H stainless steel  
Cold-formed  
Welded  
Proof tested

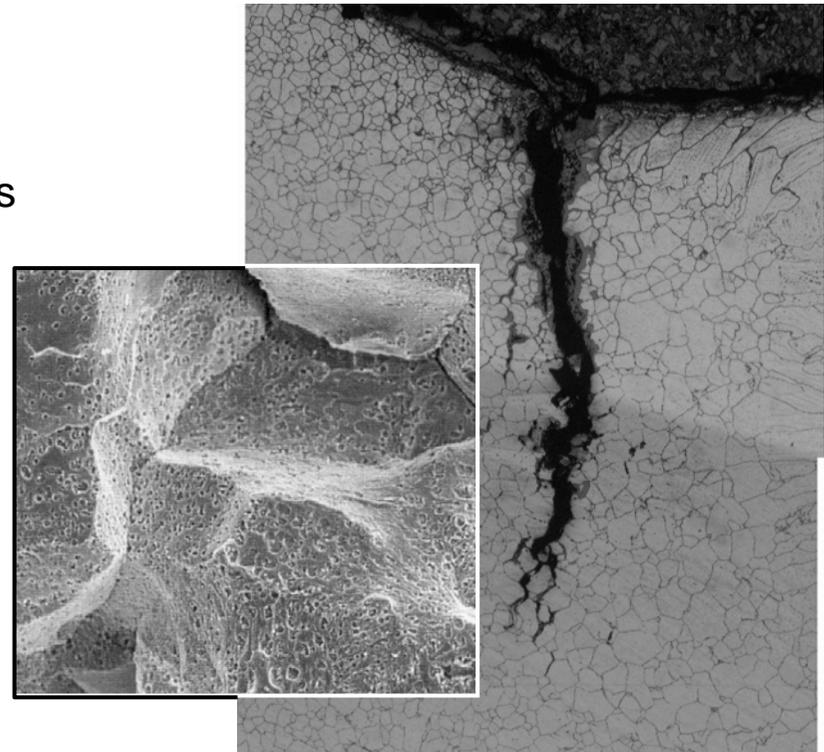
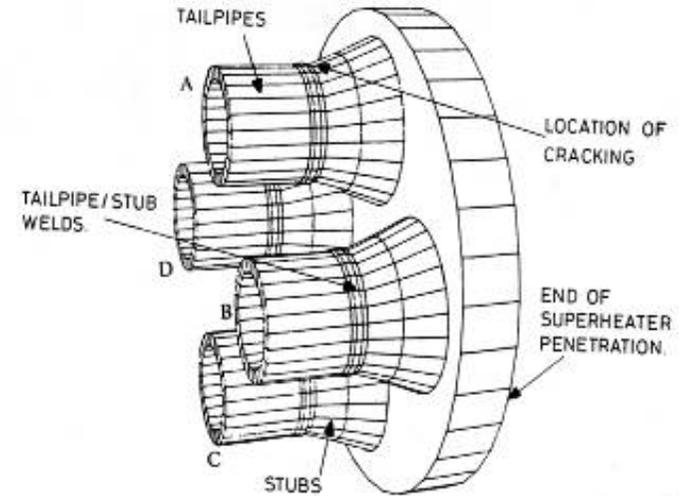
Bifurcation

**Reheater Tubing**  
(Hinkley 'B')

# AGR Plant life extension

## 2. Degradation of high temperature welds

- Degradation mechanism relates to:
  - Accumulation of creep strain due to relaxation of weld residual stresses
  - Formation of grain boundary creep cavities within the heat- and strain-affected zone of non-stress relieved welds
  - Linkage of cavitation leads to micro and then macrocracks



# AGR Plant Life Extension

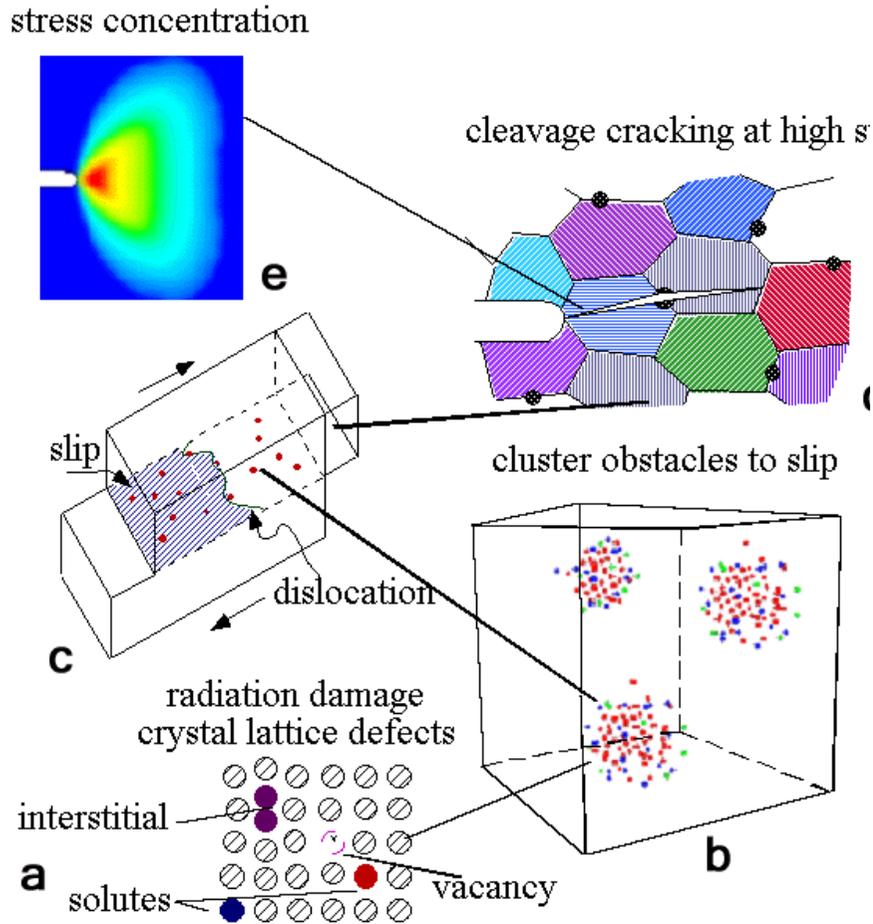
## 2. Degradation of high temperature welds

- The materials challenge includes
  - Measurement, modelling and treatment of residual/secondary stresses and associated strains
  - Long-term effects of ageing, irradiation, history (manufacturing and in-service) on creep ductility
  - Creep-fatigue damage evaluation (initiation)
  - Creep-fatigue crack assessment (growth)
  - Multiaxial stress effects creep ductility and consequent effect on fracture toughness (constraint)

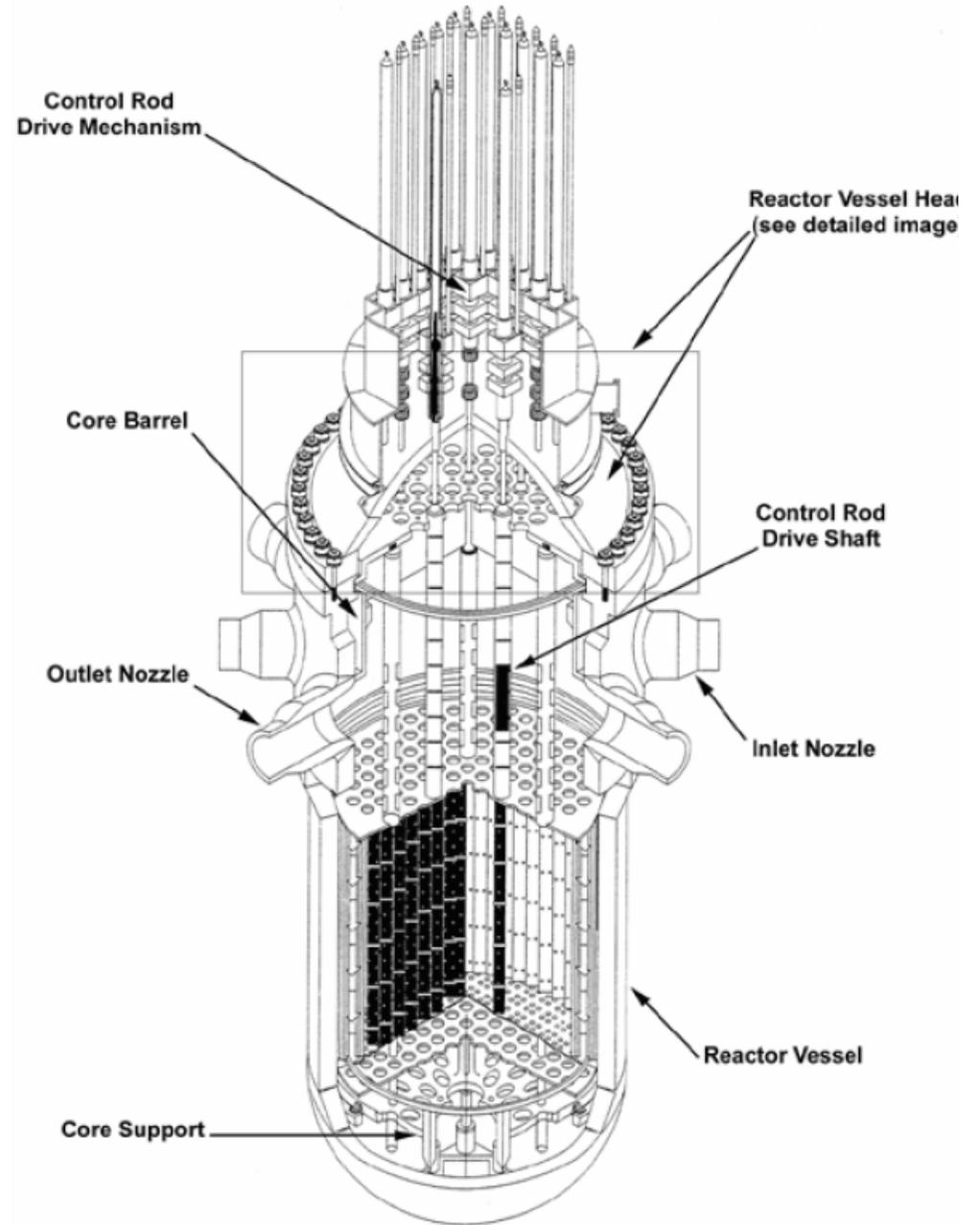


# PWR Plant Issues

## 3. RPV Embrittlement



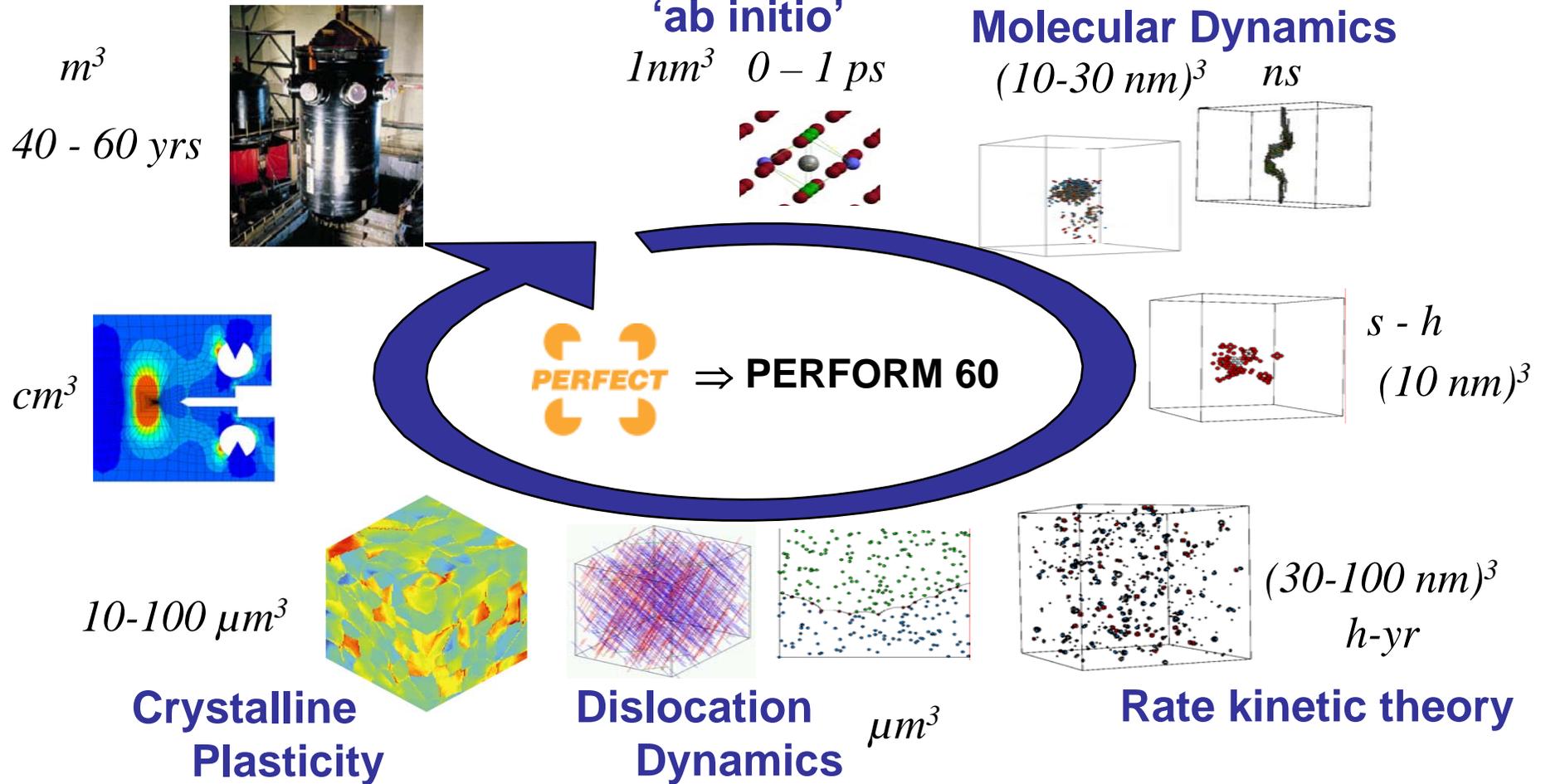
## Typical Pressurized Water Reactor



# PWR Plant Issues

## 3. RPV Embrittlement

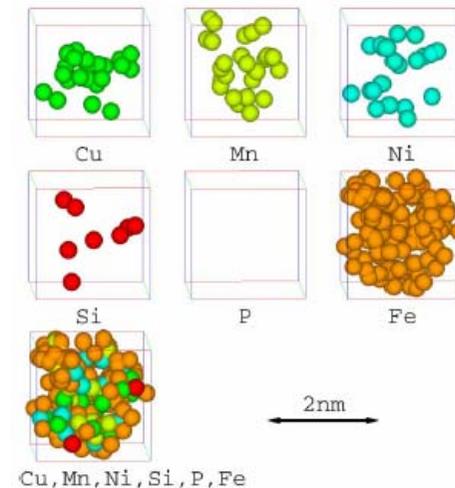
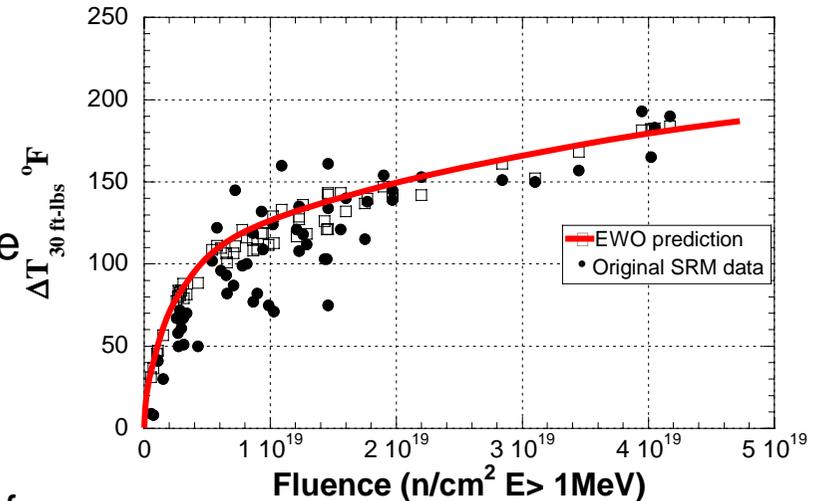
### Finite elements



# PWR Plant Issues

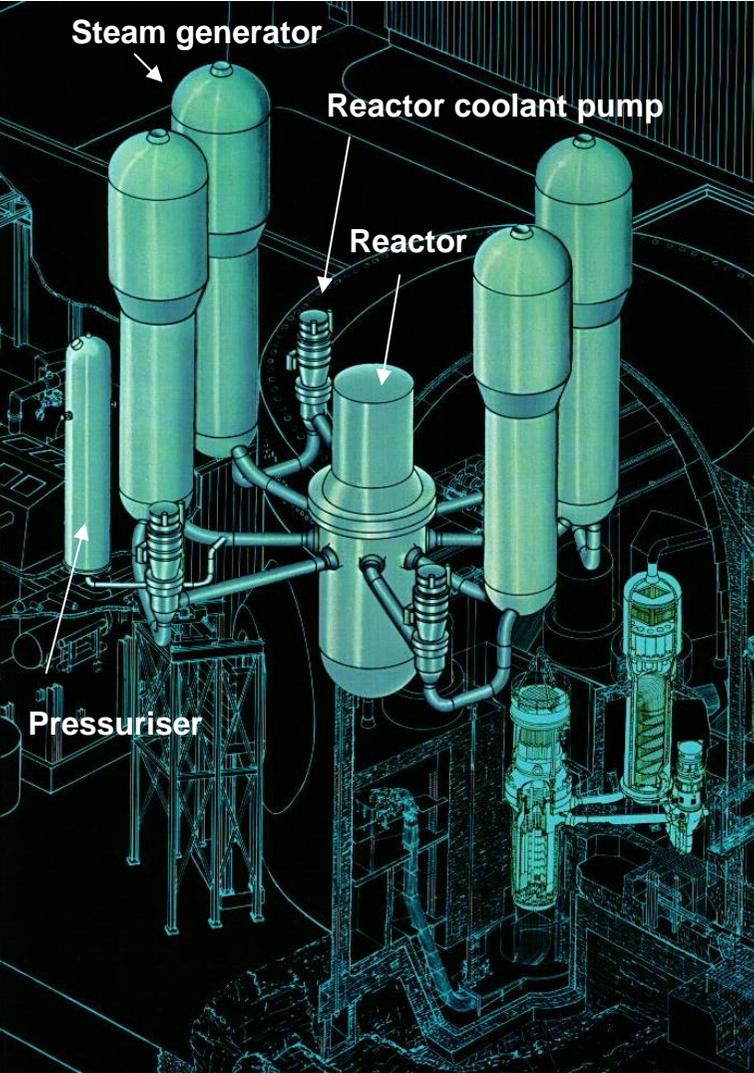
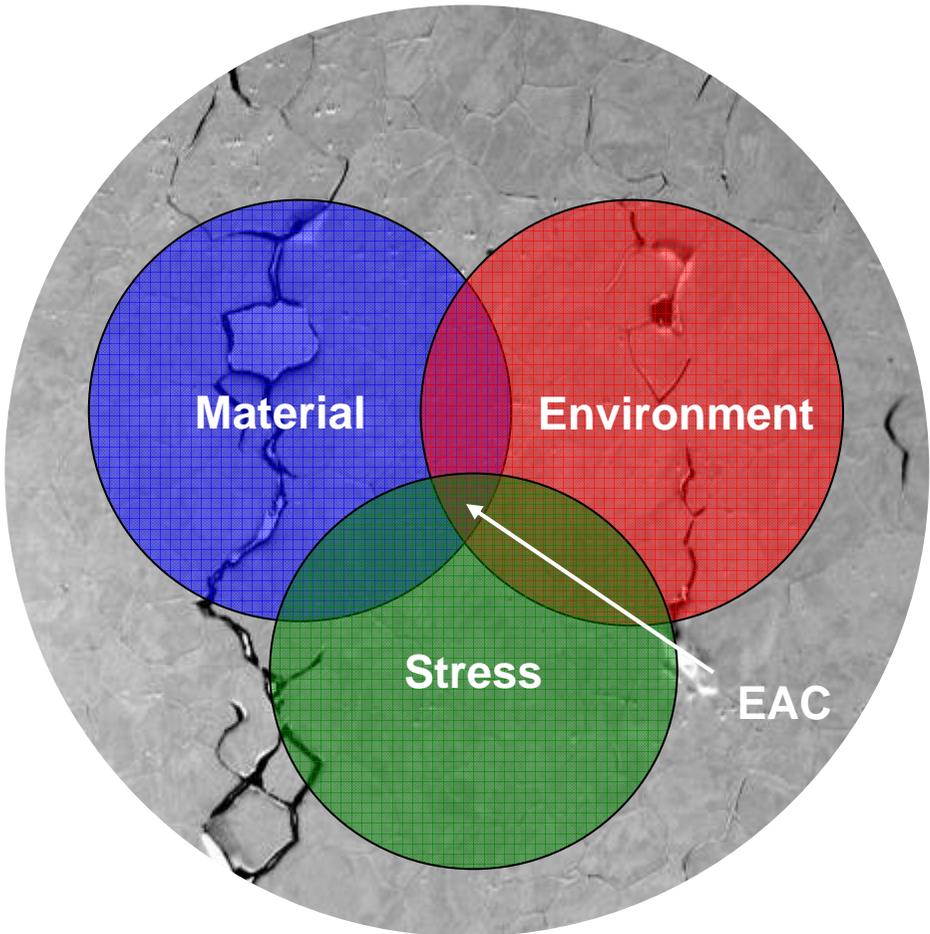
## 3. RPV embrittlement

- The materials challenges include:
  - The measurement and modelling of neutron irradiation on the microstructure and properties of RPV materials
- Development of mechanistically based correlations that predict embrittlement of operating vessels, e.g. LWRs in the USA
- High resolution microscopy and atom probe studies to assess so-called “late-blooming phase” development
- Development of multi-scale models that link atomic-scale damage to component properties



# PWR Plant Issues

## 4. Environmentally-assisted cracking



# PWR Plant Issues

## 4. Environmentally-assisted cracking

- In depth assessment of materials degradation of PWR components undertaken in 2006
- Expert Panel assessed ~ 50 groups, ~350 components and 11 degradation mechanisms
- Highest priority issues ranked:
  - Susceptibility index
  - State of current knowledge
- Many of the highest priority issues related to EAC in PWR water

NUREG/CR-6923

Subgroup Description	Degradation Mechanisms										
	BAC	CREEP	CREV	FAC	FAT	FR	IC	PI	SCC	SW	WEAR
<b>Type 304/316/308 SS Socket Welds</b>											
1.7											
2.7											
3.7											
5.6											
6.6											
7.6											
<b>Type 308/309 SS Dissimilar Metal Welds</b>											
1.9											
2.9											
3.9											
<b>Inconel Alloy 82/182 Dissimilar Metal Welds</b>											
4.6											
10.8											
11.16											
<b>Inconel Alloy 600 Components</b>											
4.7											
4.14											
10.9											
11.5											
11.6											
11.9											
11.12											
11.14											
11.22											
11.23											
<b>High-Strength Components</b>											
9.3											
12.7											
12.12											
<b>Carbon and Low-Alloy Steel Components</b>											
10.2											
11.20											
<b>Type 304/316/308 SS Components</b>											
10.10											
12.4											
12.8											
12.9											
12.10											
12.11											

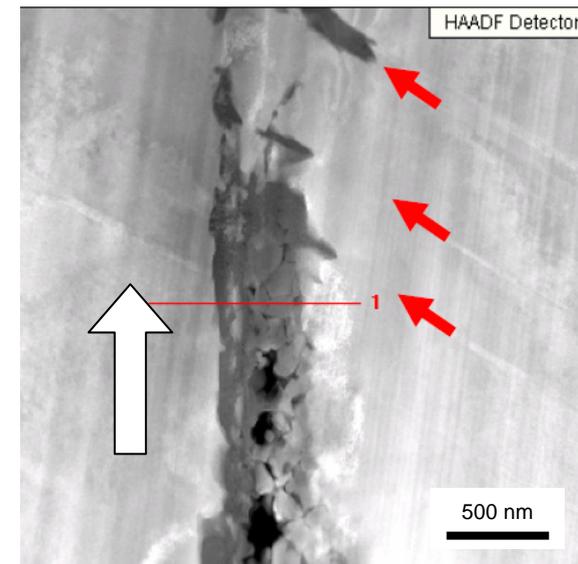
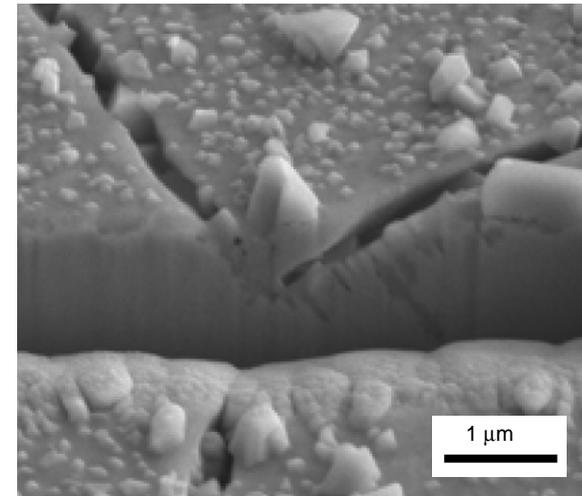
NOTES: \* Susceptibility at color interface with one or more scores higher than this interface; \* Susceptibility inside color box with one or more scores higher than color box interface.

Figure 3.6 Modified Rainbow Chart Showing Red Subgroups in PWR Reactor Coolant System

# PWR Plant Issues

## 4. Environmentally-assisted cracking

- The materials challenges include:
  - development of improved mechanistic understanding and predictive models
    - SCC in non-sensitized stainless steels where cold work increases susceptibility
    - Corrosion-fatigue in high temperature water
    - Irradiation-assisted SCC including the effect of radiolysis, deformation mechanisms and sensitisation
    - Non stress-relieved welds, including dissimilar metal welds



SCC crack in CW 304 stainless steel

# Fission Materials

## Key lessons

- Nuclear graphite
  - Understanding the interaction between changes in microstructure, stress state and properties has improved predictive capability
- Boiler Welds
  - Understanding distribution of residual stress and strain on high temperature welds has improved assessment of where and when degradation will occur
- RPV Embrittlement
  - Improved mechanistic understanding at the atomic level has improved predictive methodologies that influence the operation of PWR plant.
- EAC
  - Experimental approach has provided valuable data to assess plant susceptibility but highlights need for mechanistic understanding of cold work, corrosion-fatigue and irradiation effects on EAC.

# Fission Materials Roadmap

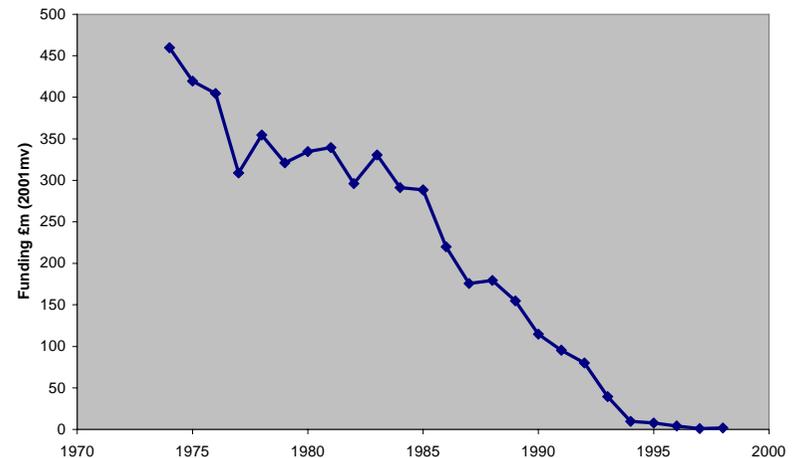
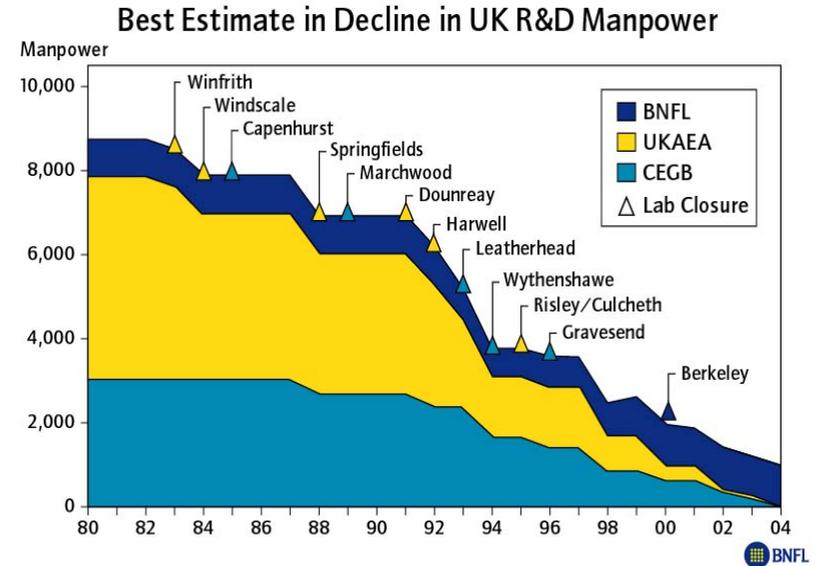
National Nuclear Goals	Short-term (2009-2015)			Medium-term (2015-2020)			Long-term (2020-2060)		
	Life ext'n AGRs	Selection of Gen III+ reactors	Decom. Reproc. Storage Disposal	Emergent issues Gen III+ reactors	Design & materials for Gen IV	Decom. Reproc, Storage Disposal	Life ext'n Gen III+ issues Gen IV	Design & materials for fusion reactors	Decom. Reproc, Storage Disposal.

# U.K. Nuclear Research Capability

## Fission materials skills

- Significant skills challenge to support the nuclear power renaissance
- The decline in U.K. nuclear R&D manpower has led to an impoverished research community.
- Nuclear materials research has not been supported within nuclear organisations

To establish a stable critical mass of research expertise within key technology areas with the necessary continuity of support



# U.K. Nuclear Research Capability

## Fission materials facilities

- Significant capability challenge to support the nuclear power renaissance
- The closure of U.K nuclear research laboratories has led to relatively few key facilities which are scattered widely across the U.K. – fragmentation.
- The use of fragmented and expensive facilities has reduced – utilisation.

To establish a network of key facilities and access arrangements necessary to deliver solutions to priority nuclear materials research issues



# U.K. Nuclear Research Capability

## What has changed recently?

- Clear benefit from developments in:
  - New training and research programmes
    - Postgraduate education (NTEC)
    - Research Council funding (KNOO, EngD)
    - Industry-University partnerships, e.g. NDA/NNL URAs, BE University Partnerships)
    - Naval propulsion programme
  - Facilities development including
    - Establishment of UK NNL
    - Manchester-NDA £20m investment in new facilities in study radiation sciences



# Fission Materials

## Meeting the challenge

- Increase and maintain funding for fission materials R&D to address short, medium and long-term national nuclear goals
- Build and sustain partnership between academic institutions, industrial stakeholders and Government to:
  - Provide strategic focus to ensure nuclear goals are met through targeted R&D
  - Enhance the aggregation and utilisation of existing national nuclear facilities and the creation of new facilities, where necessary
  - Connect the best materials scientists and structural engineers within the UK and overseas to deliver and deploy research outputs in a timely manner
  - Ensure benefit gained from knowledge transfer
  - Expand skills development and career pathways for nuclear materials scientists and structural engineers
- Enhance international cooperation in research and skills development through European Framework programmes and targeted strategic links with Nuclear Centres of Excellence worldwide

